NUCLEAR FUSION **PRINCIPLES AND PROGRESS**

Maxim Mai

University of Bonn

08.03.2024 Habilitationskolloquium



MM@ChatGPT4.0

OUTLINE

I. Motivation

II. Fundamental principles

nuclear reactions, electrostatic repulsion, reaction rates, plasma, ...



III. Technical implementation

- Lawson criterium,
 - ignition,
 - confinement,
 - fuel self-sufficiency, ...



IV. Summary



MM@ChatGPT4.0





Electricity production

	known since	availability	impact on nature
fossil fuels	~1880s	flexible (60-900y)	CO ² pollution
natural	~1900s	local	areal changes
renewable	~1890s	stochastic	areal changes
nuclear fission	~1950s	flexible (200y+)	nuclear waste
nuclear fusion	?	flexible(?)	~ 0(?)



Energy density^[1]



[1] International Energy Agency (IEA), Department of Energy (DOE), Nuclear Energy Agency (NEA)





Nuclear fusion in "action"

- 1. stars
 - gravity: high density, temperature $T \sim 10^7 \, {
 m K}$
 - ${}^{1}_{1}H + {}^{1}_{1}H \rightarrow {}^{2}_{1}H + e^{+} + \nu \quad \Delta E = 0.42 \text{ MeV}$
 - CNO and tripple-alpha processes fine-tuning^[1]
- 2. fission/fusion weapons
 - fission stage: high density, temperature $T \sim 10^8 \,\mathrm{K}$
 - fusion stage: deuterium-tritium reaction

[1] Adams, Phys. Rept. 807 (2019) 1; Lähde et al. Eur.Phys.J.A 56 (2020) 3 [2] Dittmar, Energy 37 (2012) 35-40



fission weapons \leftrightarrow commercial reactors = $\mathcal{O}(10y)$

... can one de-weaponise fusion?^[2]

II. FUNDAMENTAL PRINCIPLES

Power balance

electrostatic repulsion

Ignition

Lawson criterion

. . .



MM@ChatGPT4.0

BASIC PRINCIPLE

Nuclear fusion

- $X(B_X) + Y(B_Y) \rightarrow Z(B_Z)$
- Excess in energy $\Delta E = B_Z B_Y B_X > 0$ for small A
- Example^[1]:

$$\underbrace{{}^{2}_{1}H}_{B=2.2 \text{ MeV}} + \underbrace{{}^{2}_{1}H}_{B=2.2 \text{ MeV}} \rightarrow \underbrace{{}^{3}_{2}\text{He}}_{B=7.8 \text{ MeV}} + {}^{1}n \quad (\Delta E = 3.3 \text{ MeV})$$



Strong(short-) vs. EM(long-range) force

(eV)





COULOMB BARRIER

Reactants are charged!

- Electrostatic repulsion
 - 1. deuterium-deuterium ${}_{1}^{2}H + {}_{1}^{2}H \rightarrow {}_{2}^{3}He + {}^{1}n + 3.3 \text{ MeV}$
 - 2. deuterium-deuterium ${}_{1}^{2}H + {}_{1}^{2}H \rightarrow {}_{1}^{3}H + {}_{1}^{1}H + 4.0 \text{ MeV}$
 - 3. deuterium-tritium ${}_{1}^{2}H + {}_{1}^{3}H \rightarrow {}_{2}^{4}He + {}^{1}n + 17.6 \text{ MeV}$
 - 4. deuterium-helium3 ${}_{1}^{2}H + {}_{2}^{3}He \rightarrow {}_{2}^{4}He + {}_{1}^{1}H + 18.3 \text{ MeV}$
- Tunnelling: larger distances (lower temperature)
 - ... through WKB^[1]



v • **• •** 7







Reactants do not have fixed momentum

• Maxwell-averaged cross section $\langle \sigma v \rangle$

 \rightarrow reaction rate $n_1 n_2 \langle \sigma v \rangle$

- R-matrix parametrisation^[1] (see also NCSM^[2], NLEFT^[3]...)
 - 1. deuterium-deuterium ${}_{1}^{2}H + {}_{1}^{2}H \rightarrow {}_{2}^{3}He + {}^{1}n + 3.3$ MeV
 - 2. deuterium-deuterium ${}_{1}^{2}H + {}_{1}^{2}H \rightarrow {}_{1}^{3}H + {}_{1}^{1}H + 4.0 \text{ MeV}$
 - ${}^{2}_{1}H + {}^{3}_{1}H \rightarrow {}^{4}_{2}He + {}^{1}n + 17.6 \text{ MeV}$ 3. deuterium-tritium
 - 4. deuterium-helium3 ${}_{1}^{2}H + {}_{2}^{3}He \rightarrow {}_{2}^{4}He + {}_{1}^{1}H + 18.3 \text{ MeV}$

[1] Bosch/Hale 1992 Nucl. Fusion 32 611

[2] Navrátil/Quaglioni *Phys.Rev.Lett.* 108 (2012) 042503; Hupin/Quaglioni/Navrátil Nature Comm. 10 351 (2019) [3] PRL 115, 122301 (2015); DD scattering in progress.. Meyer et al.

REACTION RATES

T/MK







Reactants do not have fixed momentum

- Maxwell-averaged cross section $\langle \sigma v \rangle$
 - \rightarrow reaction rate $n_1 n_2 \langle \sigma v \rangle$
- R-matrix parametrisation^[1] (see also NCSM^[2], NLEFT^[3]...)
 - 1. deuterium-deuterium ${}_{1}^{2}H + {}_{1}^{2}H \rightarrow {}_{2}^{3}He + {}^{1}n + 3.3$ MeV
 - 2. deuterium-deuterium ${}^{2}_{1}H + {}^{2}_{1}H \rightarrow {}^{3}_{1}H + {}^{1}_{1}H + 4.0$ MeV
 - ${}^{2}_{1}H + {}^{3}_{1}H \rightarrow {}^{4}_{2}He + {}^{1}n + 17.6 \text{ MeV}$ 3. deuterium-tritium
 - 4. deuterium-helium3 ${}_{1}^{2}H + {}_{2}^{3}He \rightarrow {}_{2}^{4}He + {}_{1}^{1}H + 18.3 \text{ MeV}$

➡ most favourable* reaction deuterium-tritium (DT)

energy density ($\sim 4 \cdot 10^8 \, \text{MJ/kg}$)

orders of magnitude higher than chemical (eV)

[1] Bosch/Hale 1992 Nucl. Fusion 32 611

[2] Navrátil/Quaglioni Phys. Rev. Lett. 108 (2012) 042503; Hupin/Quaglioni/Navrátil Nature Comm. 10 351 (2019) [3] PRL 115, 122301 (2015); DD scattering in progress. Meyer et al.

REACTION RATES

T/MK









III.TECHNICAL REALISATION

Confinement

Stability

Self-efficiency

. . .

Tokamak/Stellarator



MM@ChatGPT4.0



SOURCES OF ENERGY

 S_h

- External heating (ohmic, microwaves):
- Fusion reaction $DT \rightarrow \alpha(3.5 \text{ MeV}) + n(14.1 \text{ MeV})$ *neutrons*: large mean free path \rightarrow reactor output small mean free path $\mathcal{O}(1 \text{ mm}) \rightarrow$ plasma reheating alpha:

$$S_{\alpha} = E_{\alpha} \frac{n_{\alpha}^2}{4} \langle \sigma v \rangle = E_{\alpha} \frac{\langle \sigma v \rangle}{16} \frac{p^2}{T^2}$$

POWER BALANCE RELATION^[1] $\frac{E_{\alpha}\langle \sigma v \rangle p^2}{16 T^2} + S_h = \frac{C_B}{4} \frac{p^2}{T^{3/2}} + \frac{3 p}{2 \tau_E}$ W m^3

[1] Lawson (1957). Proceedings of the Physical Society B70, 6

PLASMA EQUILIBRIUM

SINKS OF ENERGY

• **Bremsstrahlung** (dominantly electrons)

$$S_B = C_B n^2 T^{1/2} = \frac{C_B}{4} \frac{p^2}{T^{3/2}}$$

- Fluid dynamics of plasma (steady state):
 - \rightarrow Heat conduction: $S_d = \nabla(\kappa \nabla T)$. Empirically,

$$S_d = \frac{2}{3} \frac{p}{\tau_E}$$

... relaxation time τ_E .

Corrections: volume expansion/convection/micro-turbulences/ temperature profile/...



• Lawson parameter $(p\tau_E)_I = \frac{K_\kappa T^2}{K_\alpha \langle \sigma v \rangle - K_B T^{1/2}}$

... high-T solution — stable equilibrium

... low-T solution — unstable equilibrium \rightarrow burn control



$p\tau_E/(atm s)$





• Lawson parameter $(p\tau_E)_I = \frac{K_\kappa T^2}{K_\alpha \langle \sigma v \rangle - K_B T^{1/2}}$





[[]FIG] Wurzel et al. Phys. Plasmas 29, 062103 (2022)

Lawson parameter
$$(p\tau_E)_I = \frac{K_\kappa T^2}{K_\alpha \langle \sigma v \rangle - K_B T^{1/2}}$$

• External heating $S_h > 0$ can reduce $(p\tau_E)_{min}$ (reducing gain)



[FIG] Wurzel et al. Phys. Plasmas 29, 062103 (2022)

• Lawson parameter $(p\tau_E)_I = \frac{K_\kappa T^2}{K_\alpha \langle \sigma v \rangle - K_R T^{1/2}}$

• External heating $S_h > 0$ can reduce $(p\tau_E)_{min}$ (reducing gain)

World progress

Operating and proposed (*) facilities

[1] Lawson (1957). Proceedings of the Physical Society B70, 6

IGNITION

[[]FIG] Wurzel et al. Phys. Plasmas 29, 062103 (2022)

PLASMA CONFINEMENT

1. ICF Inertial Confinement Fusion (Laser, Ion-beam, ...)

- Laser induced shock waves in small pellets $\mathcal{O}(1 \text{ mm})$. Experiments since 1970s.
- + Only a small portion needs to be heated up.
- [+] Plasma self-heating, mean free path of ${}_{2}^{4}$ He : $\mathcal{O}(0.01 \text{ mm})$
- [+] Gain: 3.15/2.05 (2022NIF^[2])
- Confinement time $\mathcal{O}(20 \,\mathrm{ns})$
- Blows apart in the process

Overcoming Coulomb-barrier requires $T \sim 10^8 K$

- No materials can withstand such temperatures
- Fuel-Plasma needs to be confined/controlled^[1]
- Prospective ansätze:

... How to make a continuous process and collect energy?

Nova Laser Bay

Target

NIF "Big Foot" deuterium-tritium (DT) implosion

PLASMA CONFINEMENT

2. MCF Magnetic Confinement Fusion (Tokamak, Stellarator, ...)

- - +] Sustained self-heated plasma conditions $\mathcal{O}(10 \text{ min})^{[2]}$
 - + Semi-realistic reactor designs exist (Energy conversion..)
 - Complex configuration (shielding, ...)
 - Stability, Plasma discharges, Turbulences
 - Fuel self-sufficiency

Overcoming Coulomb-barrier requires $T \sim 10^8 K$

- No materials can withstand such temperatures
- Fuel-Plasma needs to be confined/controlled^[1]
- Prospective ansätze:

Locally quasi-neutral, high-conductivity (~40x copper) Plasma. Charged particles in the Plasma move along the B-field lines

ITER (Tokamak)

Wendelstein7-X (Stellarator)

a and a to go to be a construction of the second of the good state of the second of the good state of the second state of the

• Good news:

DT fusion has large energy density ($\approx 4 \cdot 10^8 \, \mathrm{MJ/kg}$)

 $\frac{\text{Consumption(Bonn)}}{\sim 16 \, \text{kg}}$ Tritium year

FUEL SELF-SUFFICIENCY

FUEL SELF-SUFFICIENCY

• Good news:

DT fusion has large energy density ($\approx 4 \cdot 10^8 \text{ MJ/kg}$) <u>Consumption(Bonn)</u> year <u>Vear</u>

• Bad news:

Concept for Tritium breeding is needed ...

Typical concept: breeding in a lithium blanket

$${}_{3}^{6}\text{Li} + n \rightarrow {}_{2}^{4}\text{He} + {}_{1}^{3}\text{H} + 4.78 \text{ MeV}$$

 ${}_{3}^{7}\text{Li} + n \rightarrow {}_{2}^{4}\text{He} + {}_{1}^{3}\text{H} + n - 2.7 \text{ MeV}$

Technological challenges^[1]

- 1 m thick layer around 1000 m³ volume
- Neutrons need to be slowed down for efficiency
- complex configuration

. . .

- high demand on materials
- Essential experiments @ITER (20??), @DEMO (2???)

[FIG] Raffray et al. / Journal of Nuclear Materials 307–311 (2002) 21–30 [1] Rubel, Journal of Fusion Energy (2019) 38:315–329; Dittmar, Energy 37 (2012) 35-40

Progress

- Basic design well understood
- Many test reactor facilities $\mathcal{O}(100)$
- Confinement time records, e.g., $t_C = 480$ s @ Wendelstein7-X
- Net (plasma) energy gain, e.g., $Q \sim 3/2$ @NIF

[FIG] Webster, "Fusion: Power for the future", Physics Education 38, 135 (2003)

SUMMARY **NUCLEAR FUSION – ASTONISHINGLY SIMPLE & ASTONISHINGLY COMPLEX**

Progress

- Basic design well understood
- Many test reactor facilities $\mathcal{O}(100)$
- Confinement time records, e.g., $t_C = 480$ s @ Wendelstein7-X
- Net (plasma) energy gain, e.g., $Q \sim 3/2$ @NIF

Challenges

- Plasma stability
- Net (engineering) gain $Q_E \sim 0$

... efficient thermal-electric conversion

• Shielding vs. very high energy, flux neutrons

... radioactive waste $T_{1/2} \sim \mathcal{O}(10 \text{y})$

- Tritium-breeding concept very theoretical:
 - ... experiments/design updates will take decades

A mechanical engineer with many books in his backpack walking a very windy road MM@ChatGPT4.0

Physical gain factor

$$Q = \frac{P_{out} - P_{in}}{P_{in}} \approx 5 \frac{p\tau_E}{(p\tau_E)_I - p\tau_E}$$

- no heating $Q = \infty$
- no fusion Q = 0

Engineering gain factor

$$Q_E = \frac{P_{out}^{el} - P_{in}^{el}}{P_{in}^{el}} \approx 2 \frac{p\tau_E - 0.37(p\tau_E)_I}{(p\tau_E)_I - p\tau_E}$$

- Depends on efficiency (absorption, heating, Li-breeding)
- "Break-even" Q=0

POWER BALANCE

TRITIUM BREEDING CHALLENGES

Radiatio hv

Exfoliation of steel caused by high-dose irradiation

TIMELINES

FIG. 2. Target gain vs calendar date. The horizontal labels mark the beginning of each year. The color of the narrow target gain bars represents different implosion designs, and the dashed horizontal line represents the target gain = 1 per the NAS ignition criteria [26].

Inner Poloidal field coils (Primary transformer circuit)

Wendelstein7-X

Plasma Instability: Z-Pinch

https://www.youtube.com/watch?v=gwOrbr8KWDs&list=PLbhKQRV6Toq4ocE3C1EwVbeY4ofwrPLn_&index=5

Plasma Physics Reports, 2021, Vol. 47, No. 8, pp. 814-825

Plasma Instability in a mirror device

Quantity	Symbol	Limiting value
Electric power output	P_{E}	1000 MW
Maximum wall loading	P_{W}	4 MW/m^2
Maximum magnetic field	$B_{\rm max}$	13 T
Maximum mechanical stress	$\sigma_{ m max}$	300 MPa $\approx 3000~atm$
Velocity-averaged cross section	$\langle \sigma v \rangle$	$3 \times 10^{-22} \text{ m}^3/\text{s}$
Fast neutron slowing down cross section	$\sigma_{ m sd}$	1 barn
Slow neutron breeding cross section in Li^6	$\sigma_{ m br}$	950 barns at 0.025 eV

Table 5.2. Basic engineering and nuclear physics constraints

Quantity	Symbol	Value
Blanket-and-shield thickness	b	1.2 m
Coil thickness	С	0.79 m
Minor radius	a	2.0 m
Major radius	R_0	5.0 m
Aspect ratio	R_0/a	2.5
Plasma surface area	A_P	400 m^2
Plasma volume	V_P	400 m^3
Power density	$\left(P_{\alpha}+P_{n}\right)/V_{P}$	4.9 MW/m ³
Magnetic field at $R = R_0$	B_0	4.7 T
Plasma pressure	р	7.2 atm
Plasma temperature	Т	15 keV
Plasma number density	n	$1.5 \times 10^{20} \text{ m}^{-3}$
Energy confinement time	$ au_{ m E}$	1.2 s
Normalized plasma pressure	β	8.2%

Table 5.3. Summary of parameters for a generic fusion reactor

